

SHINE Chemistry Overview

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SHINE Medical Technologies

- SHINE Medical Technologies is dedicated to being the world leader in safe, clean, affordable production of medical tracers and cancertreatment elements
- SMT and its partners have developed a system that can produce reactor-grade medical isotopes without a nuclear reactor
- Technology has two key aspects
 - Primary neutrons created by high-output D-T source
 - Neutrons enter an LEU solution where they multiply sub-critically and create medical isotopes
- Initial construction will produce nationally relevant quantities of Mo-99 and other medical isotopes (50% of U.S. Mo-99 demand)

Argonne's Role in Supporting SHINE



Major tasks

- Preparation of the uranyl sulfate target solution
- Development and design of the Mo-recovery system using TiO₂ sorbent
- Use of the LEU-Modified Cintichem process for Mo purification
- Periodic cleanup of irradiated target solution
- Radiation stability of system components and peroxide formation using the Van de Graaff
- Developing an understanding of radiolysis effects on
 - Solution chemistry
 - Gas generation
 - Precipitation
- Mini-SHINE experiments
- Micro-SHINE experiments

Mini-SHINE Experiments

- Argonne's mini-SHINE experiment will irradiate aqueous uranyl-sulfate solutions using an electron linac to:
 - Study the effects of fission on target-solution chemistry and radiolytic off-gas generation
 - Demonstrate the recovery and purification of ⁹⁹Mo from an irradiated target solution
 - With the assistance of PNNL, sample off gas for Xe, Kr, and I
 - Ship Mo-99 product to potential Tc-99m generator manufacturer partners
- Phase 1
- Linac will be operated initially at 35 MeV and 10 kW beam power on the target
- 5 L solution will be irradiated with neutrons generated through gamma-n reaction in tantalum target
- Maximum solution power will be \leq 0.05 kW/L
- Up to 2 Ci of Mo-99 will be produced
- Phase 2
- Experiment will be conducted at 35 MeV beam energy and up to 30 kW beam power
- 20 L solution will be irradiated with neutrons generated in a depleted-uranium (DU) target
- Maximum solution power will be \leq 0.5 kW/L
- Up to 20 Ci of Mo-99 will be produced

Mini-SHINE Progress

- Phase-1
 - Conservative approach
 - H₂O and NaHSO₄ tested first
 - To verify all system components before producing fission products
 - Radiation stability of components were verified using a Van de Graaff generator
 - Water and sodium bisulfate irradiations completed
 - 5 LEU uranyl sulfate irradiations (2 30 hours)
- Phase-2
 - Most of the equipment has been fabricated
 - Experiments to begin in November 2015

Important System Components



• 304 SS TSV with a 15-cm lightwater reflector/cooler

Shielded cell houses the TSV and Ta target

Dump tank below shielded cell stores irradiated solution



Target Solution Monitoring Glovebox

- Up to 7 samples collected during irradiation – done remotely
- Bubbles prevented reliable use of pH, conductivity, and turbidity probes
- Samples retrieved 8-24 hours post-irradiation

Mo-Recovery Glovebox

- Titania column to capture Mo-99 from irradiated uranium solution
- All operations are done remotely
 - Processing will begin 0-10 hours following irradiation
 - Target solution will be fed from the irradiation tank
 - Column effluent will go to the dump tank below the hot cell
 - Cold feeds are located inside the glovebox
 - Mo-product will exit the glovebox via a transfer line and go directly to 2nd hot cell for further processing
- Up to 15 samples can be collected from the feed, washes, and strip effluents
- Mo-product will be passed through a 2nd titania column and purified using the LEU-Modified Cintichem process

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Concentration Column and LEU-Modified Cintichem

- In a second shielded cell (bigfoot), the Mo-product solution will be concentrated by a factor of ~15 using a much smaller column
 - Mo-product from the second column will then be acidified for entry into the LEU-Modified Cintichem process
 - Mo product will be concentrated down to 50 mL
 - LEU-Modified Cintichem process will be used to purify Mo-product

Gas Analysis System

Mini-SHINE Experiment—Off-Gas Analysis and Collection

- System is kept slightly negative by a 2-pump/3-tank off gas collection system
- Off gas in monitored by use of an RGA (Residual Gas Analyzer) during operation to measure hydrogen and oxygen generation
- A catalytic convertor is in-line to recombine hydrogen and oxygen
 - No oxygen generation has been observed for several hours after startup
 - Oxygen must be bled into the system during that time to keep H₂ level to below 1%
 - Samples of the off gas are collected and being sent to PNNL for analysis of volatile fission products
 - Thus far, it appears that the major fraction of radioiodine stays in the solution during operation

Gas Collection System

- All off-gases from experiment will be collected and decay stored
- Three cylinder system with increasing pressure
 <0=>4.5=>3500 psig
- Automatically maintain pressure in the solution vessel at -3 inches of water
- Final storage 6000 psig cylinder
- Pumps inside vessels to prevent pumps leaking into atmosphere

Mo-99 Purity Specifications

 Goal is to produce 2 Ci Mo-99 that meets purity specifications for testing at GE Healthcare in the UK and 15- 20 Ci Mo-99 for testing at Lantheus Medical Imaging (phase 2)

	Product	
Ratio (X/ ⁹⁹ Mo)	Specification	
¹³¹ I/ ⁹⁹ Mo	\leq 5×10 ⁻⁵	
103 Ru/ 99 Mo	\leq 5×10 ⁻⁵	
¹³² Te/ ⁹⁹ Mo	\leq 5×10 ⁻⁵	
⁸⁹ Sr & ⁹⁰ Sr/ ⁹⁹ Mo	$\leq 6 \times 10^{-7}$	
$\Sigma \alpha / ^{99}$ Mo	$\leq 1 \times 10^{-9}$	
$\Sigma \gamma / ^{99}$ Mo	$\leq 1 \times 10^{-4}$	

- Total gamma results do not include Tc-99m, Mo-99, I-131, Ru-103, or Te-132 and have not been reported
- Total alpha results for Mo product met purity specifications all irradiations (<10⁻¹⁰ Ci- α /Ci-⁹⁹Mo)
- Ratios are based on activities 36 hours after EOB
- Te-132 was below detection limits for each final Mo-99 product
- Sr-89 & Sr-90 activities were based on Ba-140 activity which was below detection
 limits for each final Mo-99 product

Mini-SHINE Results

Irradiation	Time (hr)	Mo-99 produced (mCi)	Met Purity Specs ²	Overall Mo-99 Yield
1	2	70 ¹	Yes	95%
2	8	350	Yes	86%
3	32	810	No ³	94%
4	20	380	Yes	42% ⁴
5	12	190		

- 1. Insufficient mixing
- 2. Purity specifications do not include total gamma results
- Purity specifications not met for Ru change in base concentration on 1st recovery column – changed chemical form of Ru
- 4. Modifications made to Cintichem to help remove Ru from previous irradiation longer contact with KMnO₄ destroyed ABO-Mo complex

Irradiated Solution Chemistry

- No changes in redox chemistry for Mo-99 (phase 1 conditions)
- Only ~30% Te remained adsorbed on 1st titania column
- >90% Zr remained adsorbed on 1st titania column
- 1st titania column Ru(40%), Ce(15%), and Sb(5%)
- Fission products that co-eluted with Mo-product from 1st titania column – Ru, I, and Sb
- No precipitation of fission products or formation of uranyl peroxide

What's Next?

- Experiments have been delayed due to decrease in U concentration dilution event in dump tank & spill
- Experiments to resume next week
- LEU solution will be reconstituted(94 g-U/L \rightarrow 140 g-U/L)
- 2-hour irradiation will be performed to get new production rates
- 2 Ci production run with shipment to GE Healthcare in mid-September
- Few short irradiations will be performed micro-SHINE peroxide formation experiments
- Phase 1 will be removed for phase 2 installation

Phase 2 Mini-SHINE Experiments

- Phase 2 will have a single glovebox
- Windows are Pb-glass to provide additional shielding
- Cold solutions will be kept below the interior of the box
- Transfer port will be used to bring materials and samples in and out
- Valving system will be similar to phase 1 solenoid valves
- Column will be larger (3.5 cm ID X 13 cm L)
- Flow rates will be ~170 mL/min
- CV is 125 mL

Summary and Conclusions

- Mini-SHINE experiments mini-pilot plant for SHINE
- Chemistry is good no changes in redox at least for Mo-99
- After ~3-3.5 hours of irradiation, hydrogen and oxygen reach a steady state
- Purity specifications met for I-131, Ru-103, Te-132, and Sr-89/90 for all irradiations except 3rd
- Important shipment to GE Healthcare in UK in September 2015
- Important shipment to Lantheus Medical Imaging (December 2015)

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